

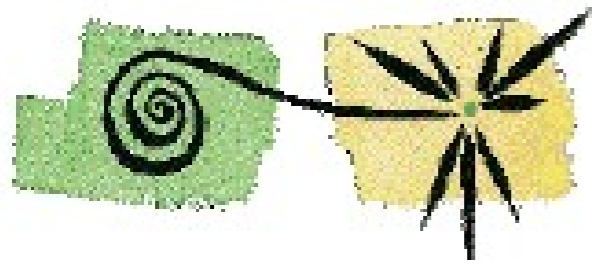
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# Safety performance of oxide and metallic fuels in SFRs

*Youpeng Zhang*

Royal Institute of Technology (KTH)





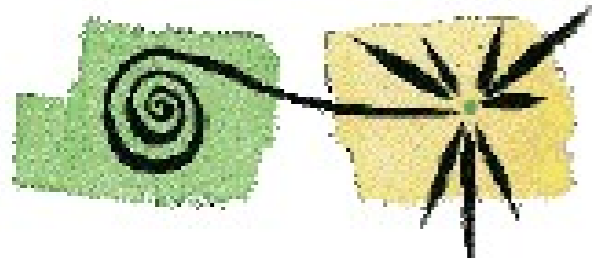
# Contents



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- **Simulation tools**
- **Implementation of the BN600 and IFR models**
- **Reactivity coefficients and core performance data**
- **Definition of transients and failure mechanisms**
- **Transient simulation results**
- **Conclusions**





# Simulation tools

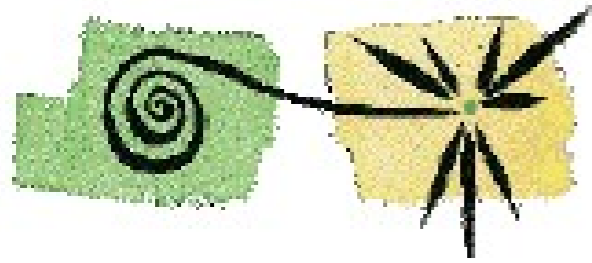


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## SERPENT Monte Carlo code

- ✓ Continuous energy Monte Carlo burnup code
- ✓ Solving Bateman equations with Transmutation Trajectory Method (TTM) for burnup calculation
- ✓ JEFF-3.1 data library
- ✓ 5 to 15 times faster than traditional Monte Carlo code due to evenly subdividing cross section energy grid





# Simulation tools

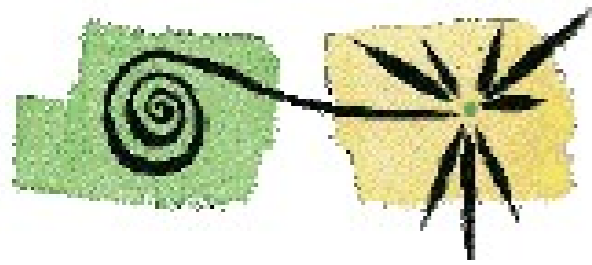


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## SAS4A/SASSYS code (ver. 3.1)

- ✓ Point kinetic model
- ✓ Two-dimensional (r/z) heat conduction equation
- ✓ One-dimensional non-equilibrium, homogenous coolant flow model
- ✓ Qualified for licensing liquid Na, liquid Pb and LBE cooled reactors loaded with oxide or metallic fuels, by means of LOF, TOP, LOHS and BT





# SERPENT models

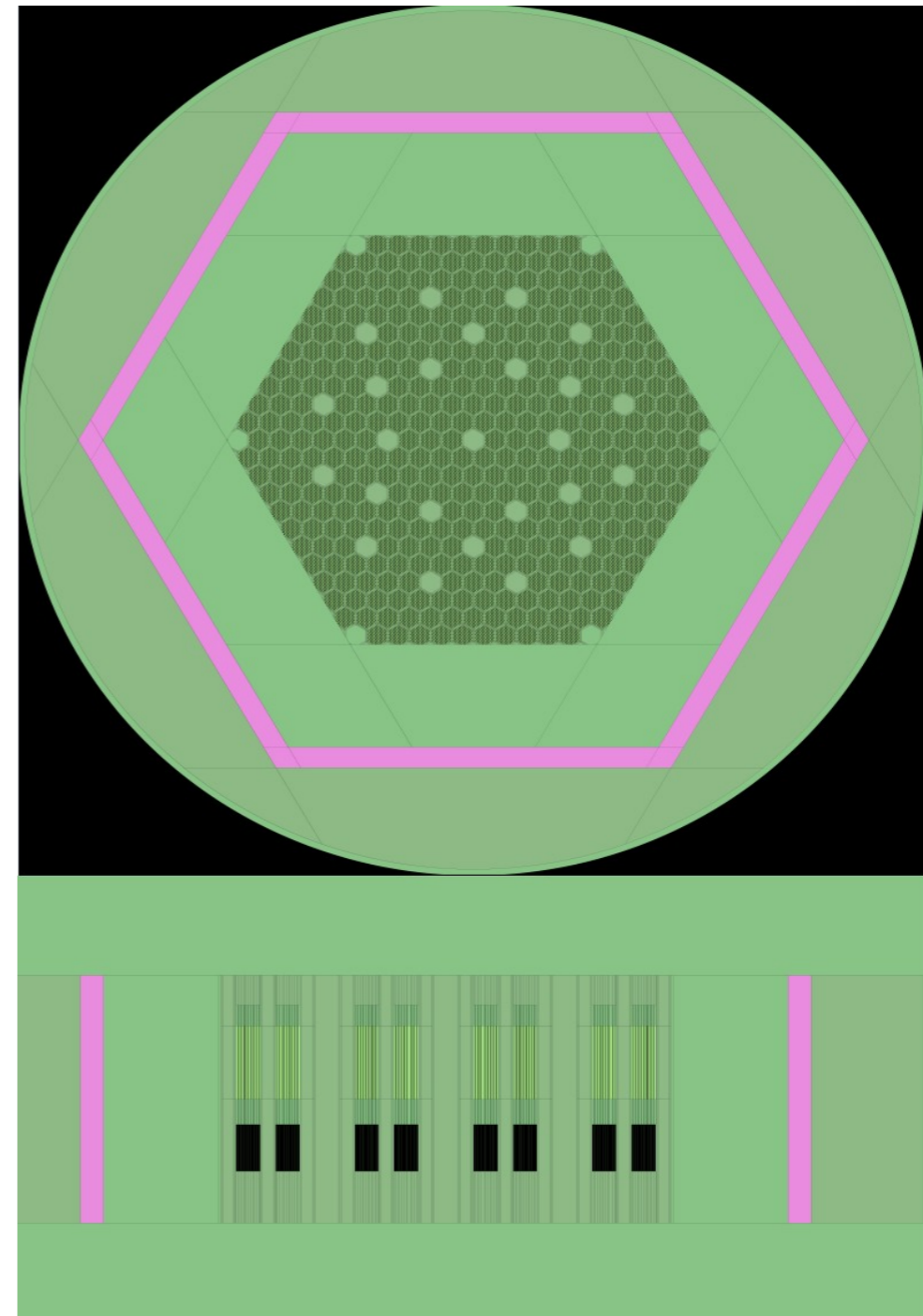


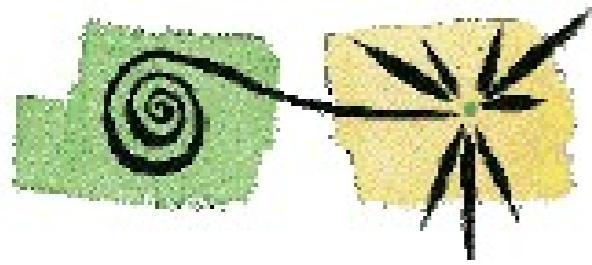
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## BN600

- 369 driver SA
- 127 pins per SA
- 8 mm pin pitch
- 99 mm SA pitch
- 20 at.% Pu + 0~10 at.% Am

Isotope	at.%	Isotope	at.%
Pu-238	3.50	Pu-239	52.88
Pu-240	23.82	Pu-241	12.90
Pu-242	7.90		
Am-241	57.58	Am-243	42.42

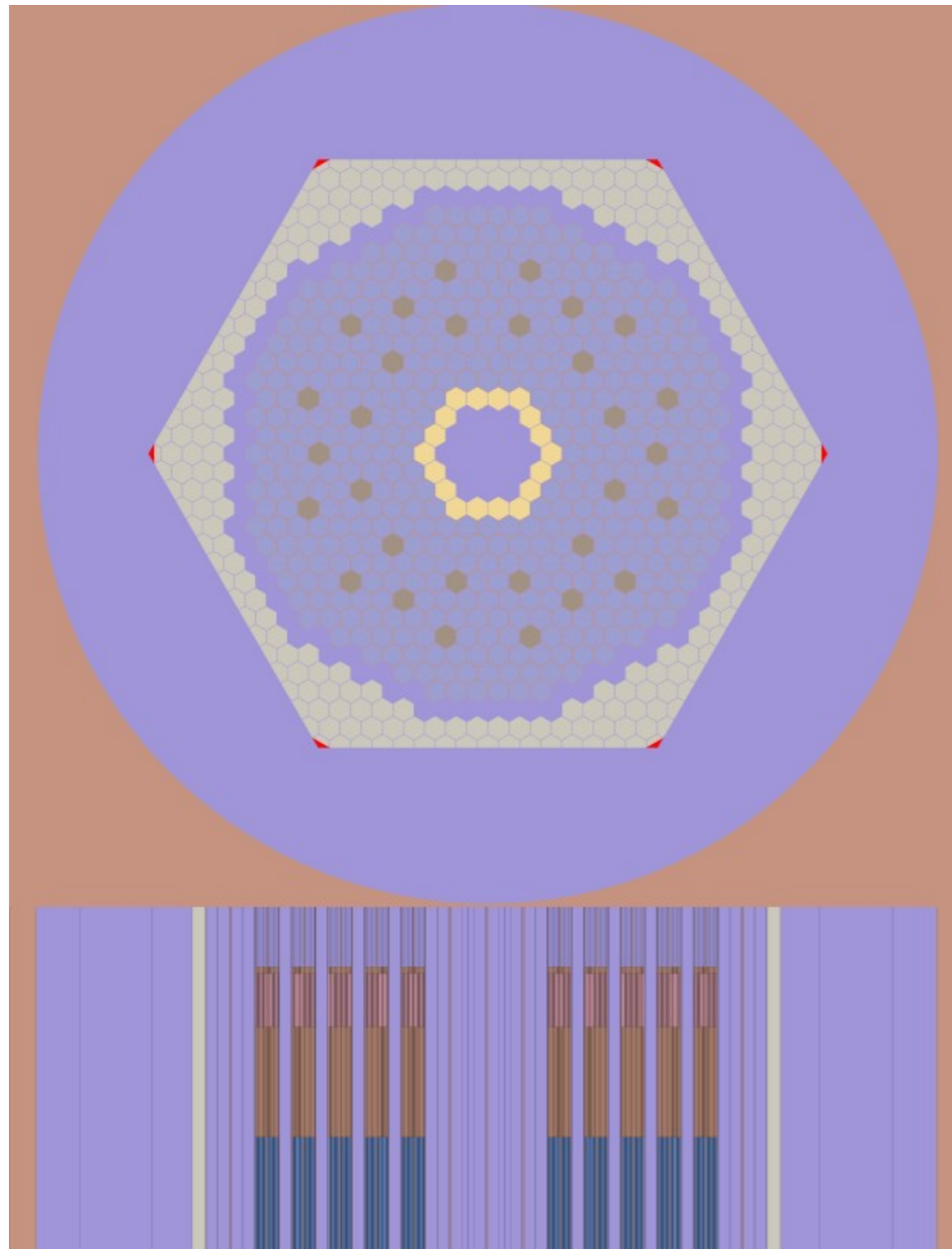




# SERPENT models



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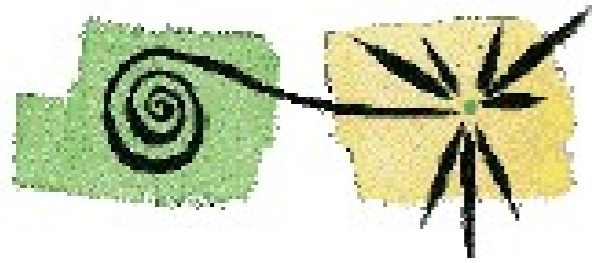


## IFR

- 456 driver SA
- 271 pins per SA
- 8.4 mm pin pitch
- 162 mm SA pitch
- 10.8 at.% Pu + 0~10 at.% Am

Isotope	at.%	Isotope	at.%
Pu-238	3.50	Pu-239	52.88
Pu-240	23.82	Pu-241	12.90
Pu-242	7.90		
Am-241	57.58	Am-243	42.42



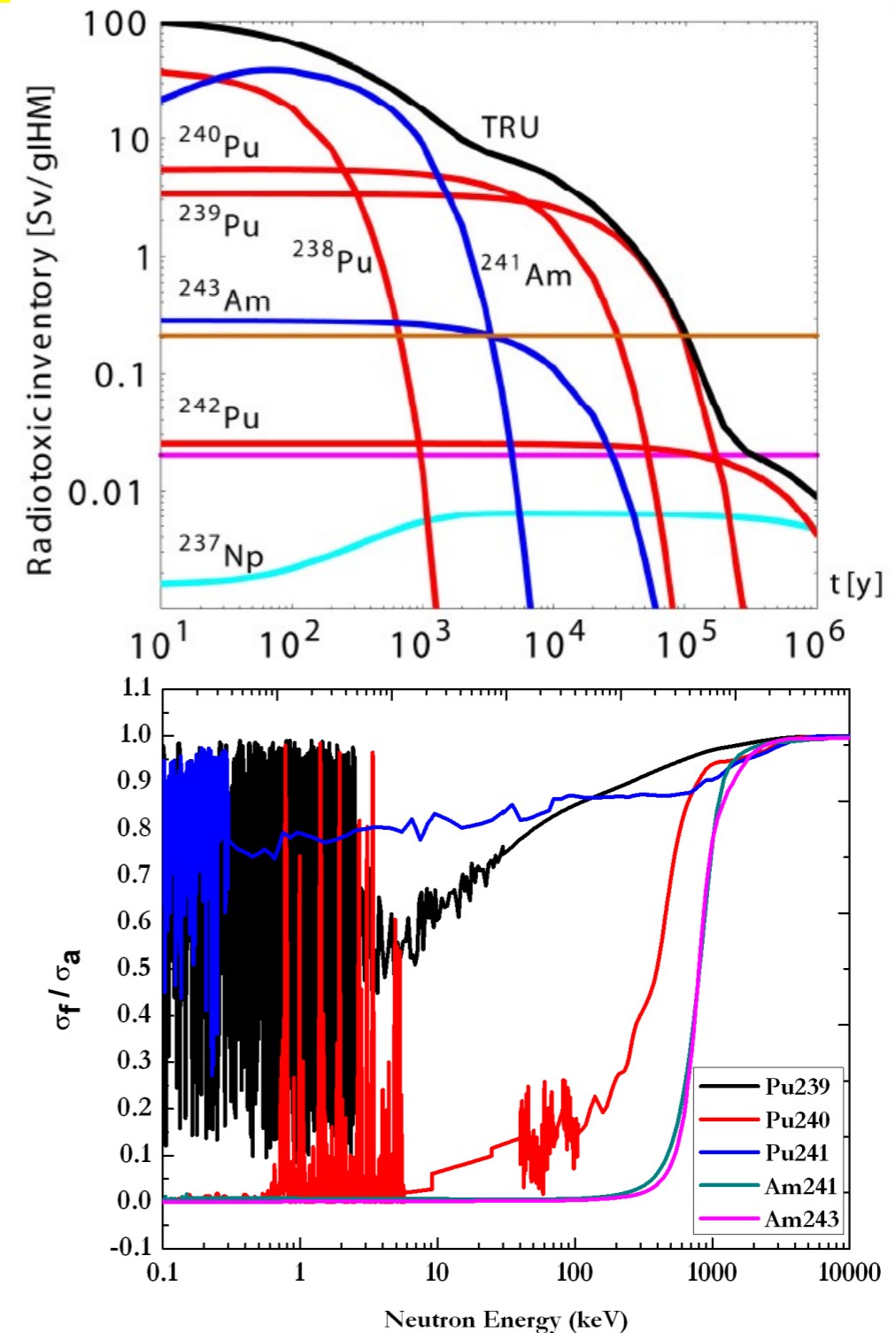


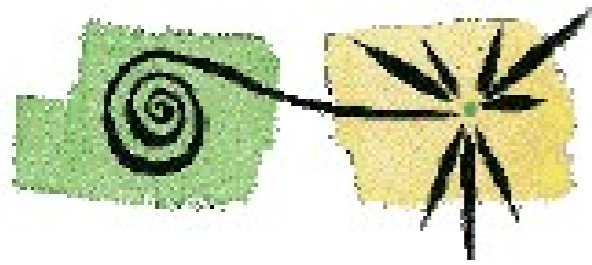
# Troublesome americium



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- Neutron emitter due to spontaneous fission reactions
- He release from alfa-decay reactions
- Could not be transmuted by multi-recycling in LWRs
- **Deterioration effects on core safety parameters**



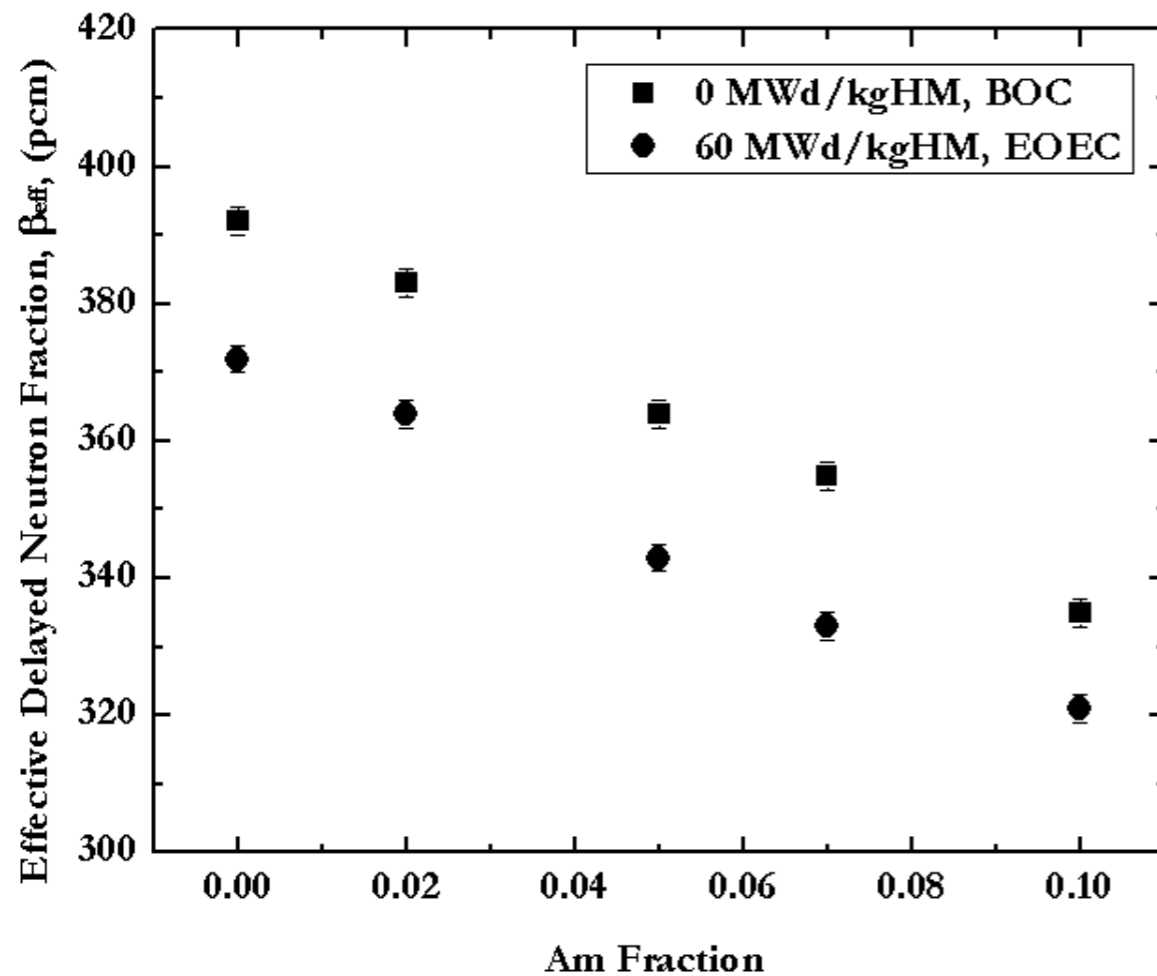


# Reactivity coefficients and core performance data

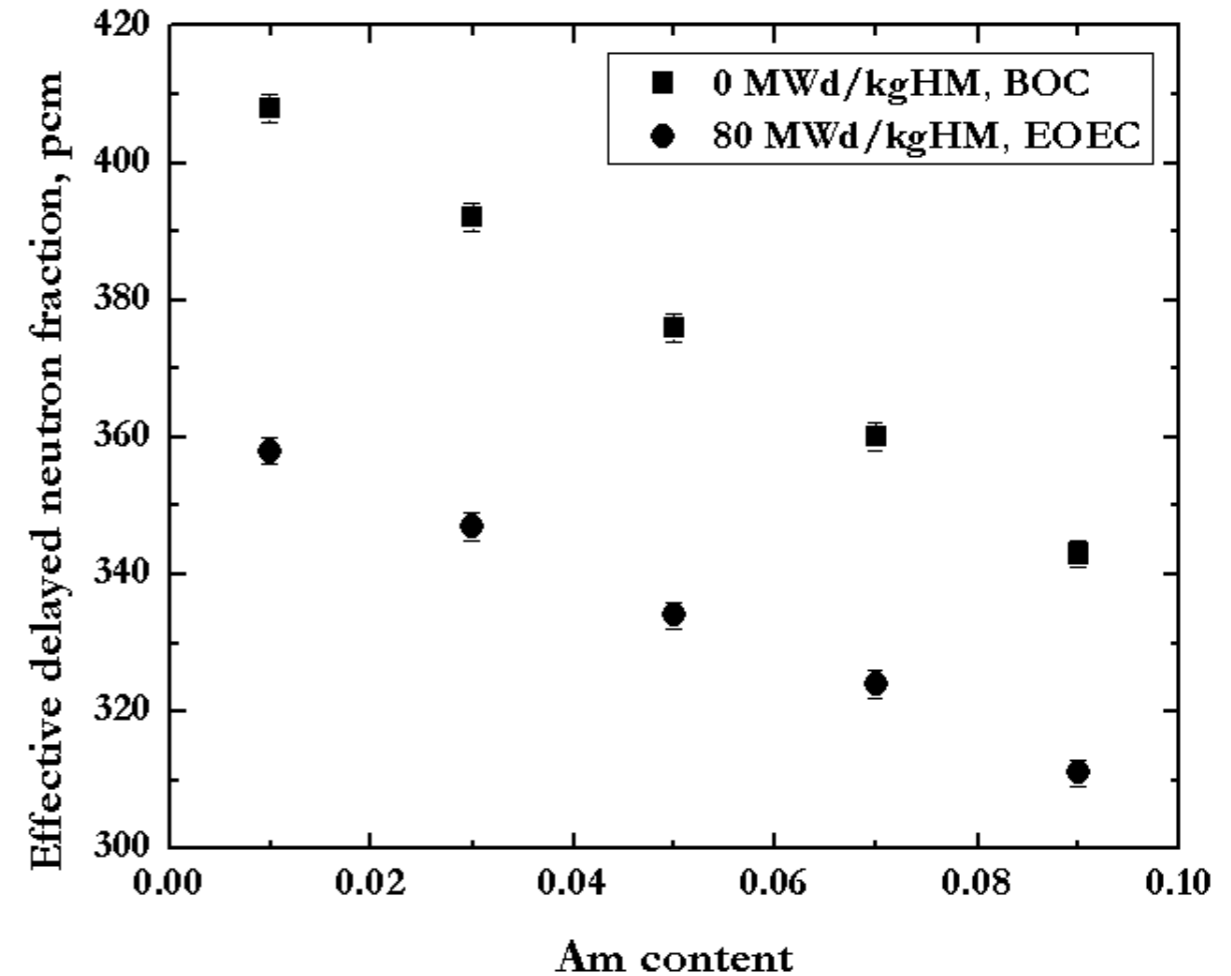


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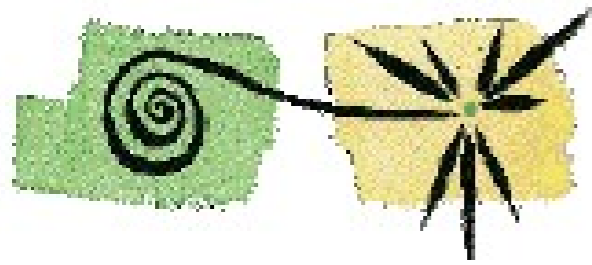
## BN600



## IFR





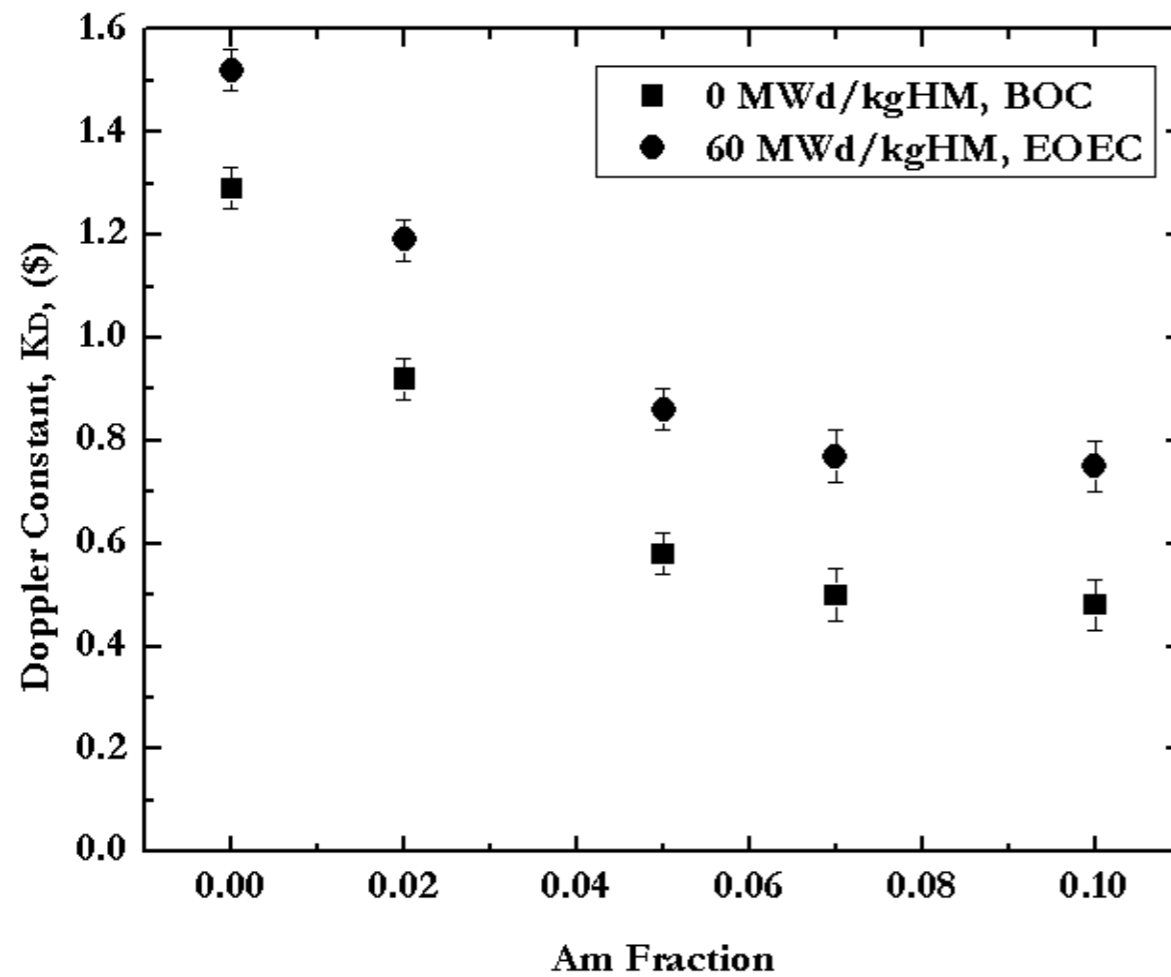


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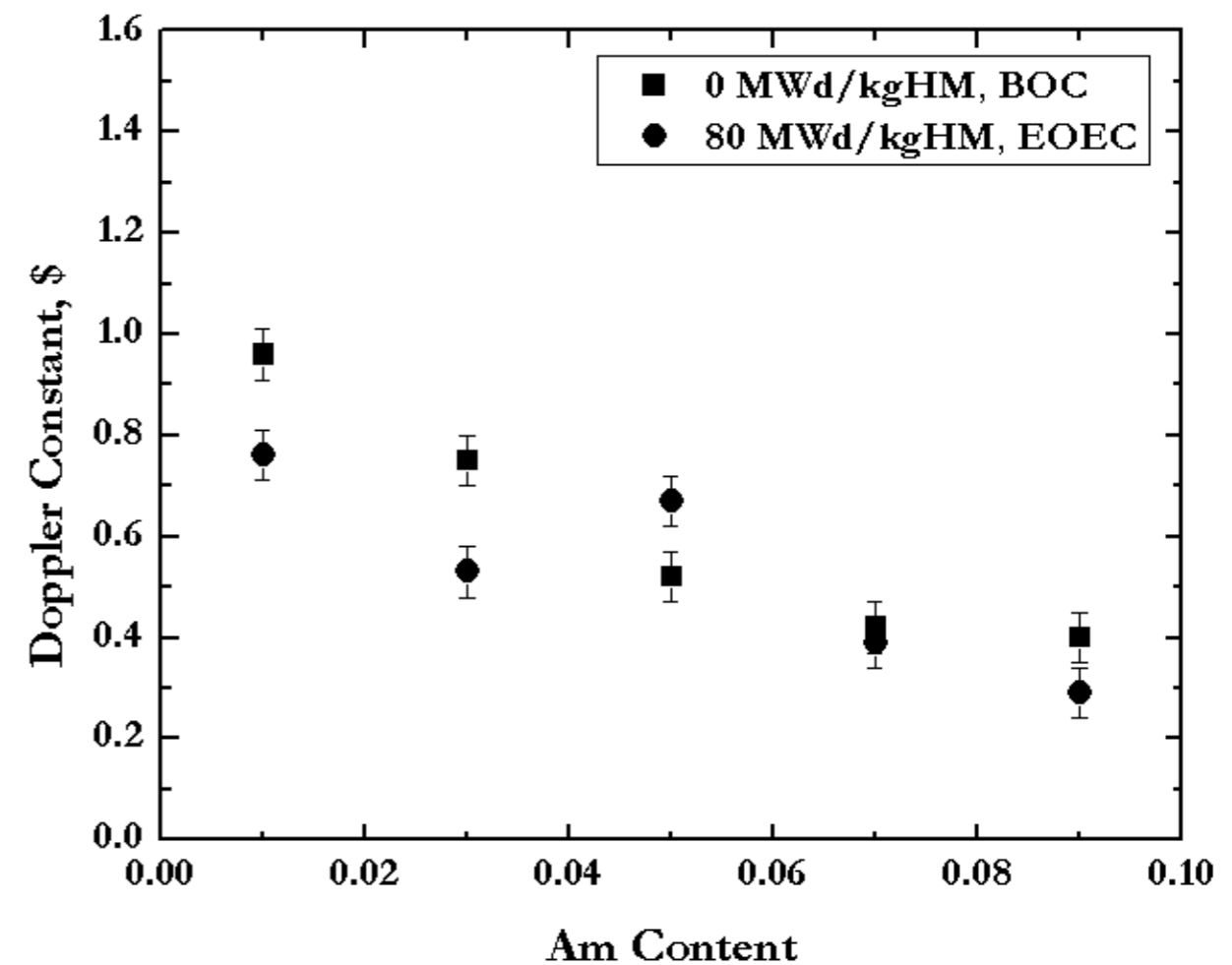


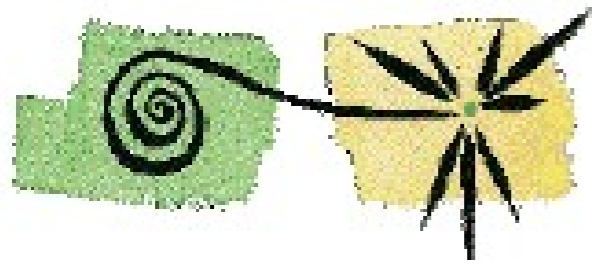
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## BN600



## IFR



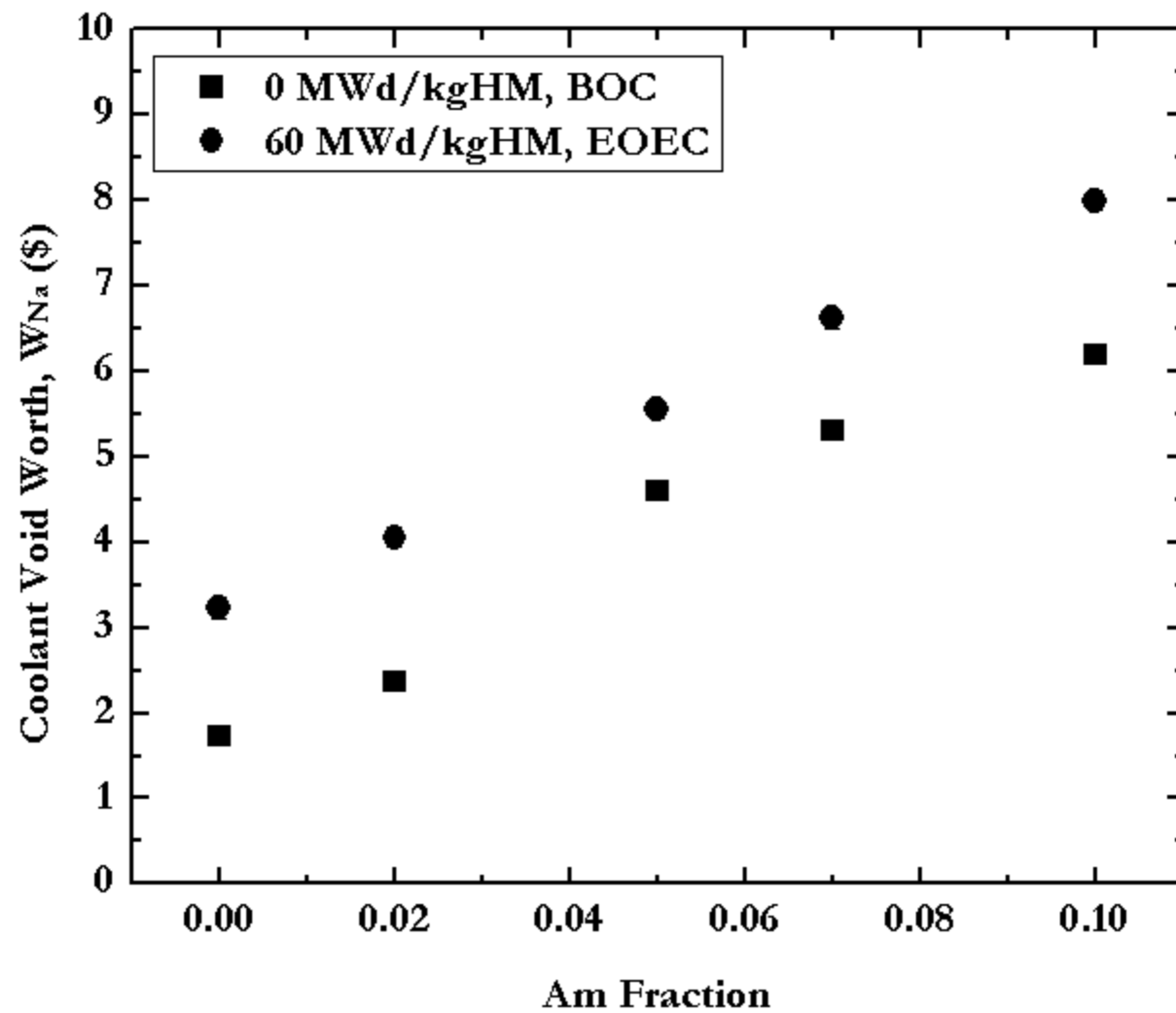


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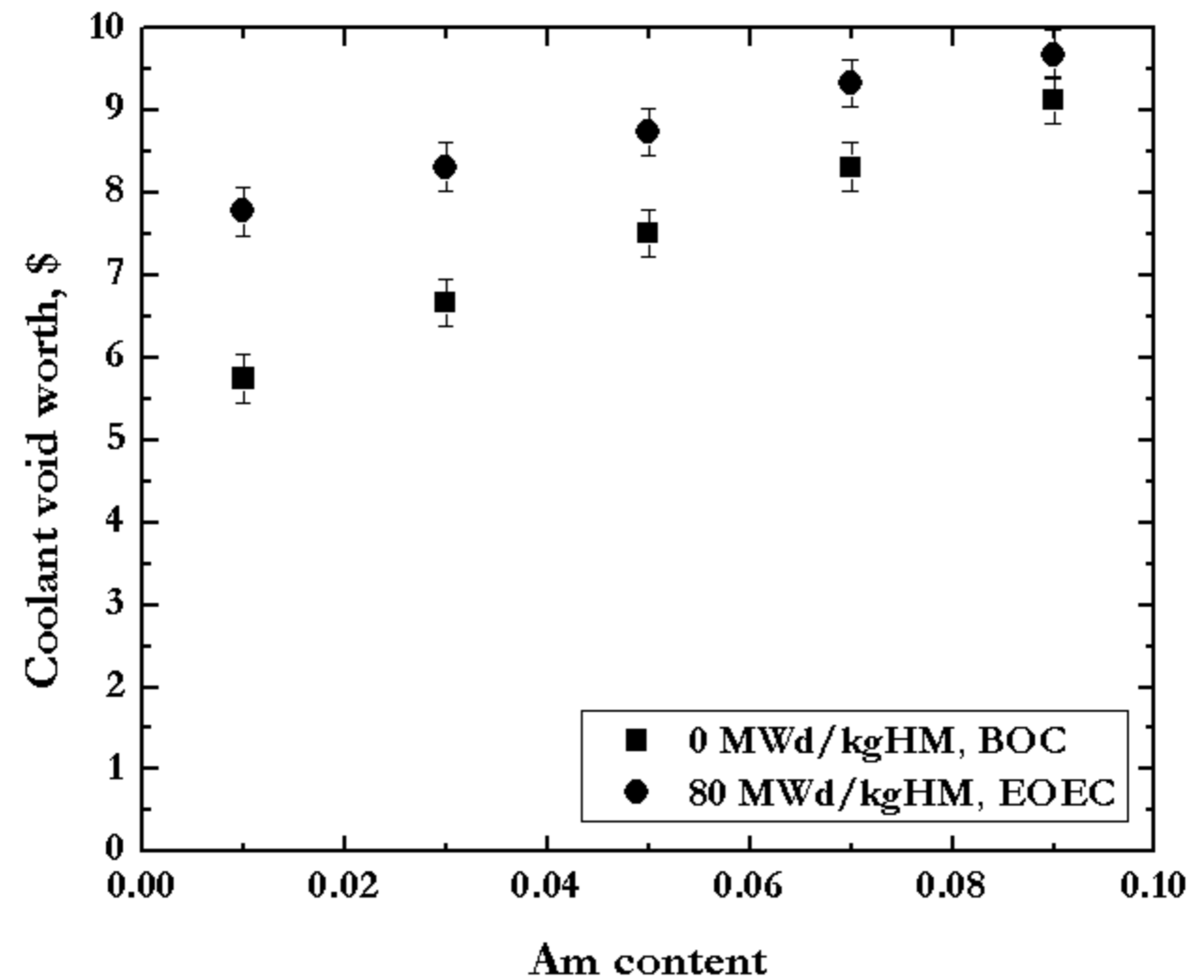


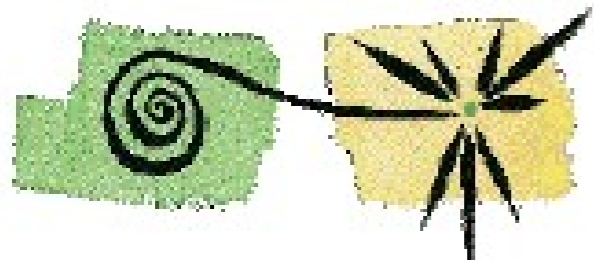
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## BN600



## IFR



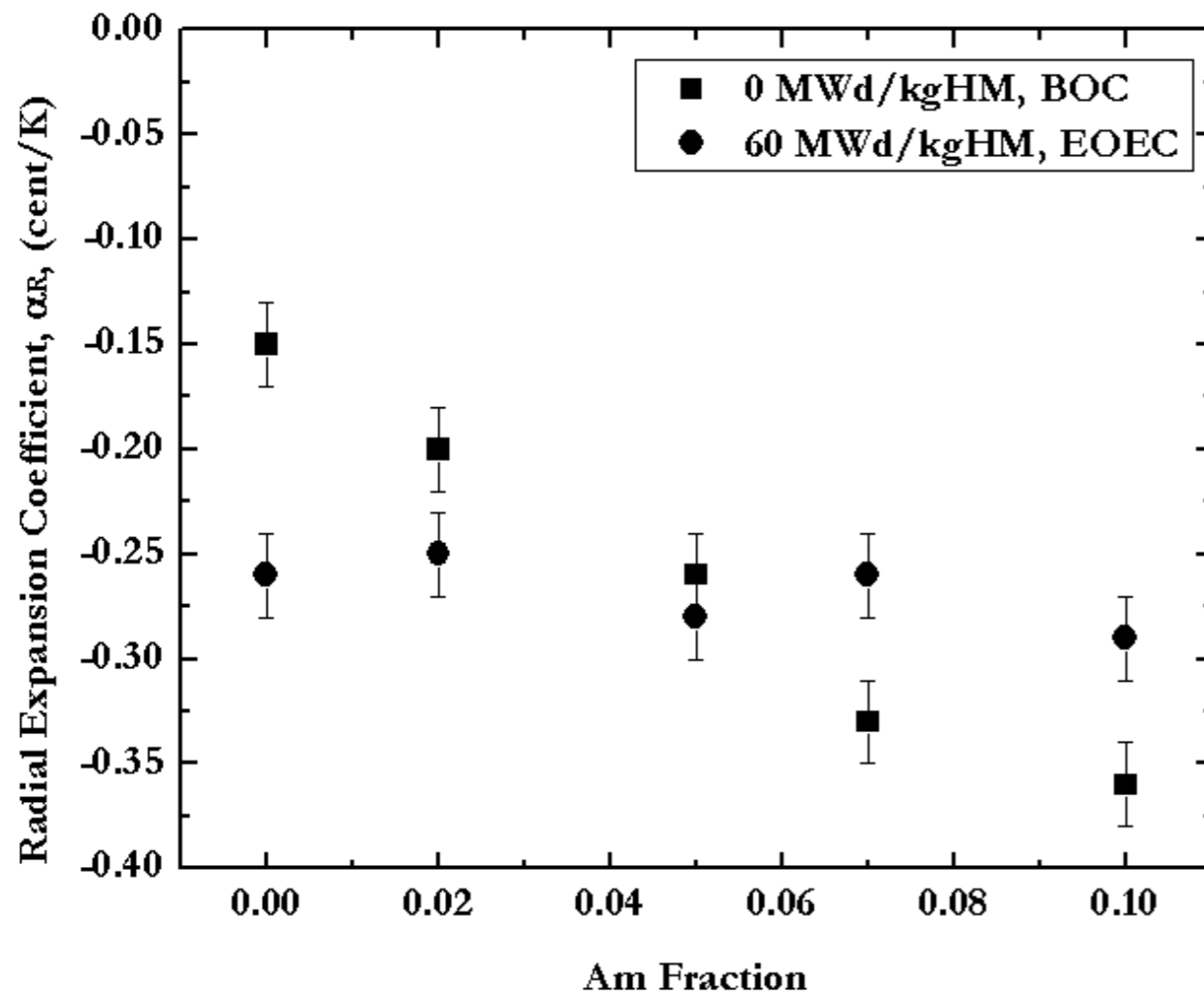


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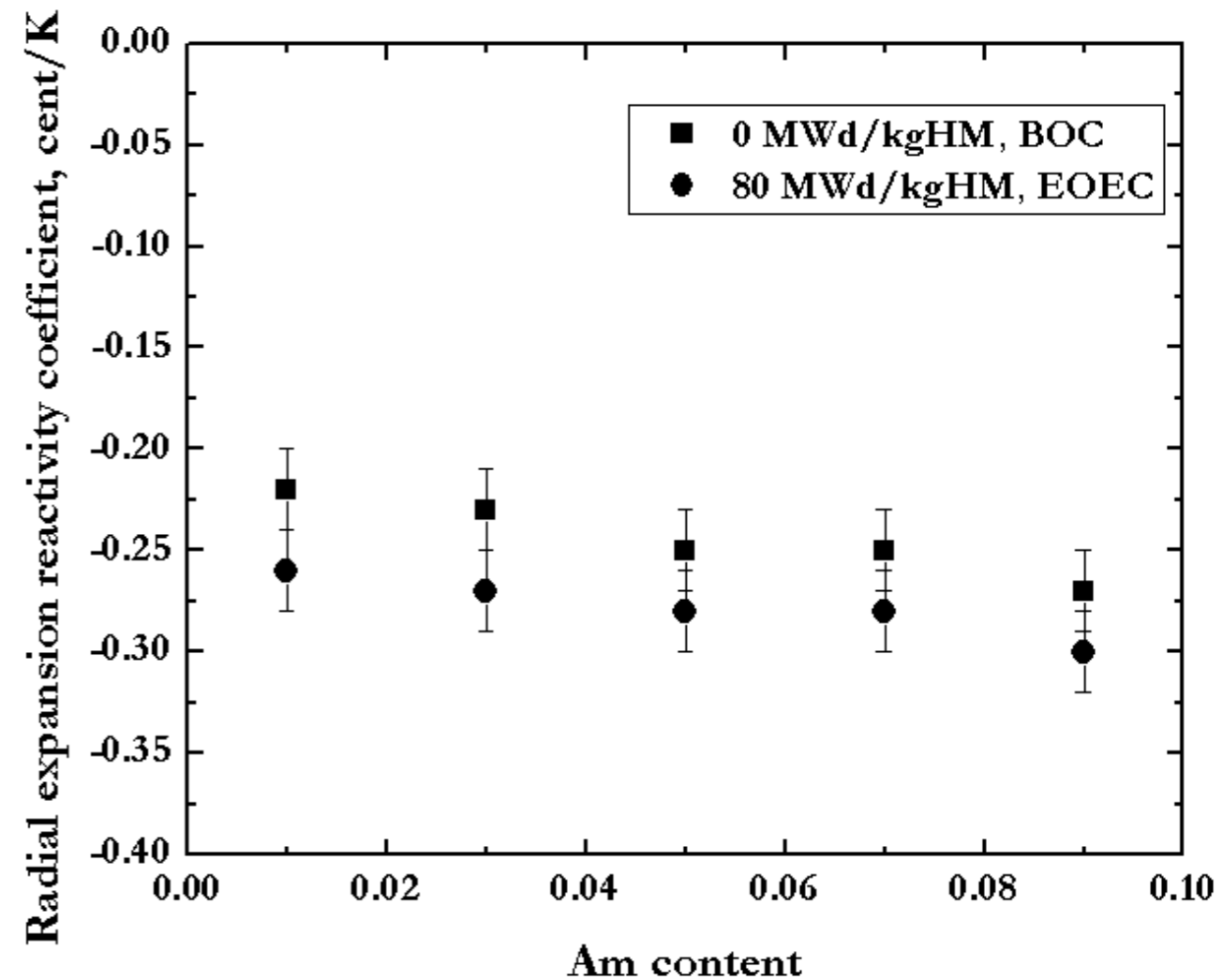


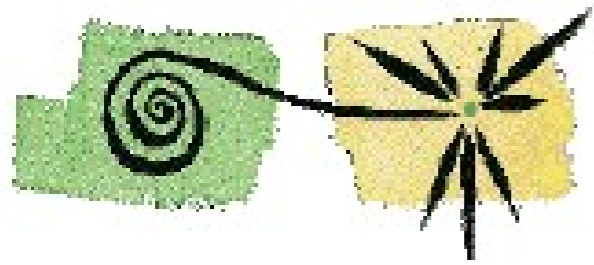
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## BN600



## IFR



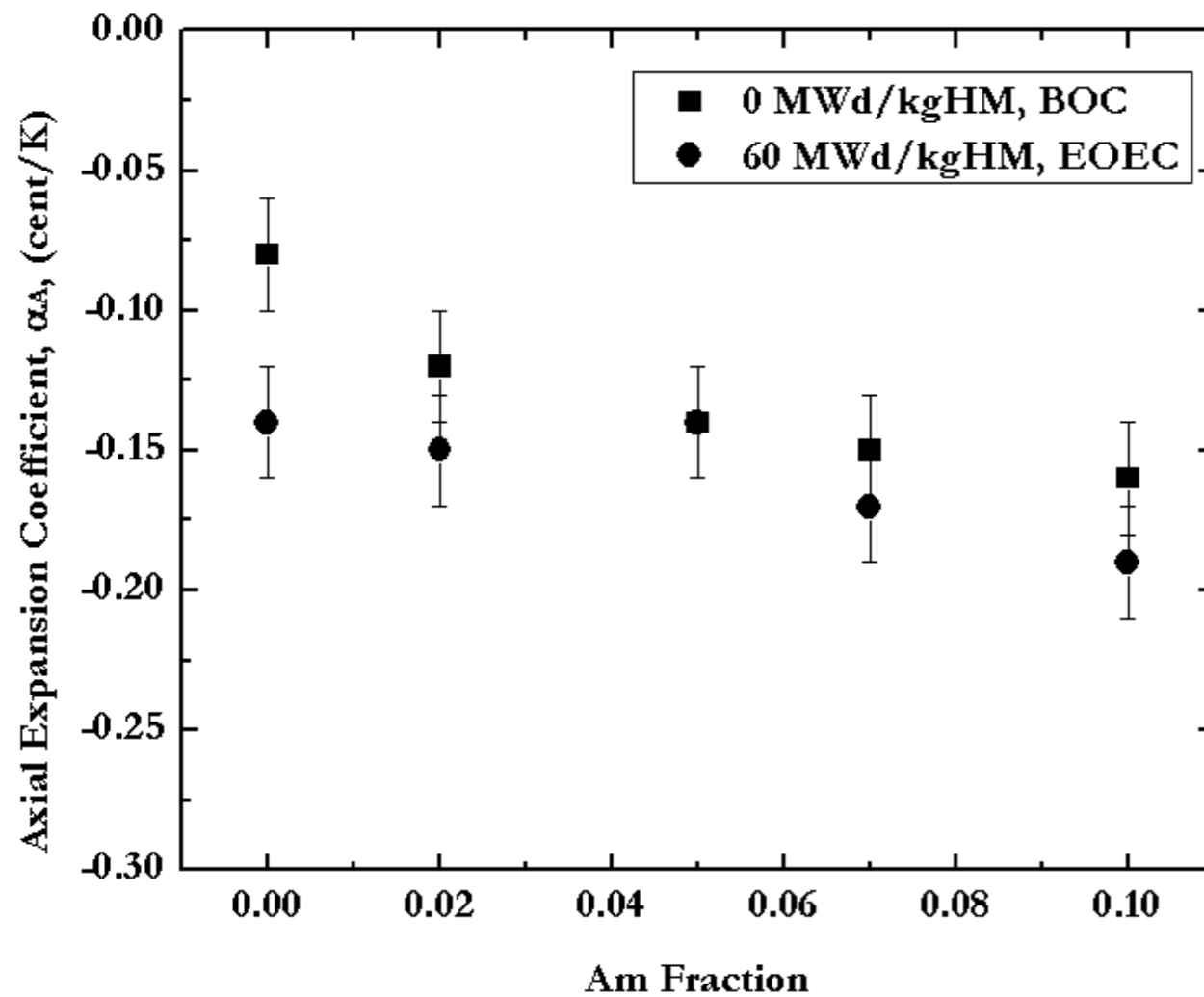


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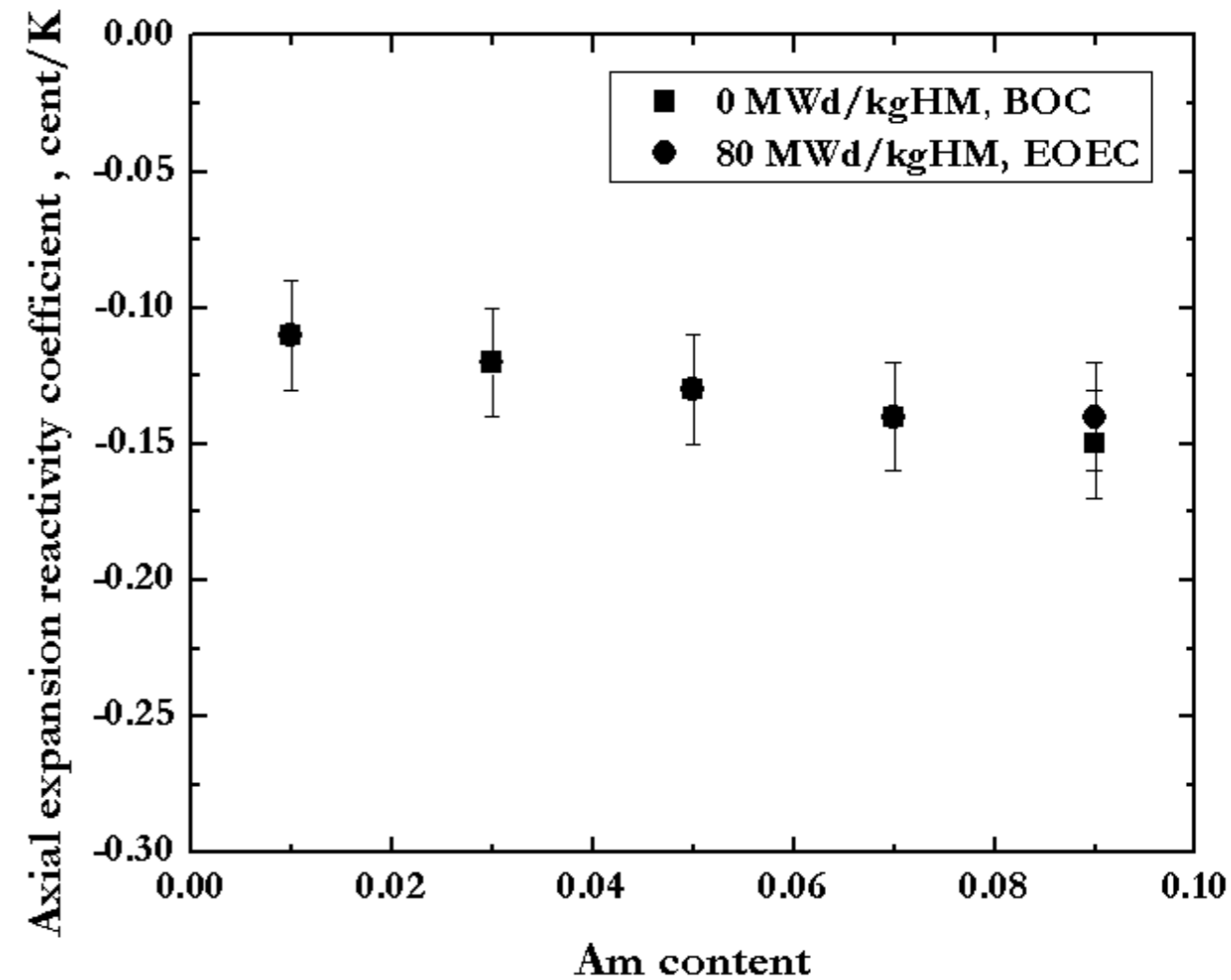


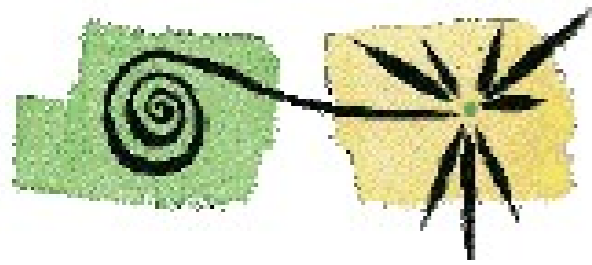
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## BN600



## IFR





# Transient definitions and failure mechanisms



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- **ULOF**

**70% coolant flowrate  
decrease within 10.0 sec.**

- **UTOP**

**1.0 \$ external reactivity  
insertion with a ramp rate  
of 0.05\$/s**

- **Cladding**

**Creep rupture – Larson-  
Miller Parameter – 1170 K  
(150 Mpa Hoop stress)**

- **Coolant boiling**

**Boiling point offset**

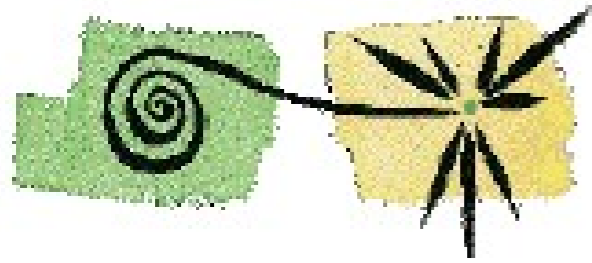
- **Fuel failure**

**Solidus temperatures offset**

✓ **2930~2951 K for oxide fuel**

✓ **1379 K for metallic fuel (U-  
Pu-Zr) due to lack of  
information**

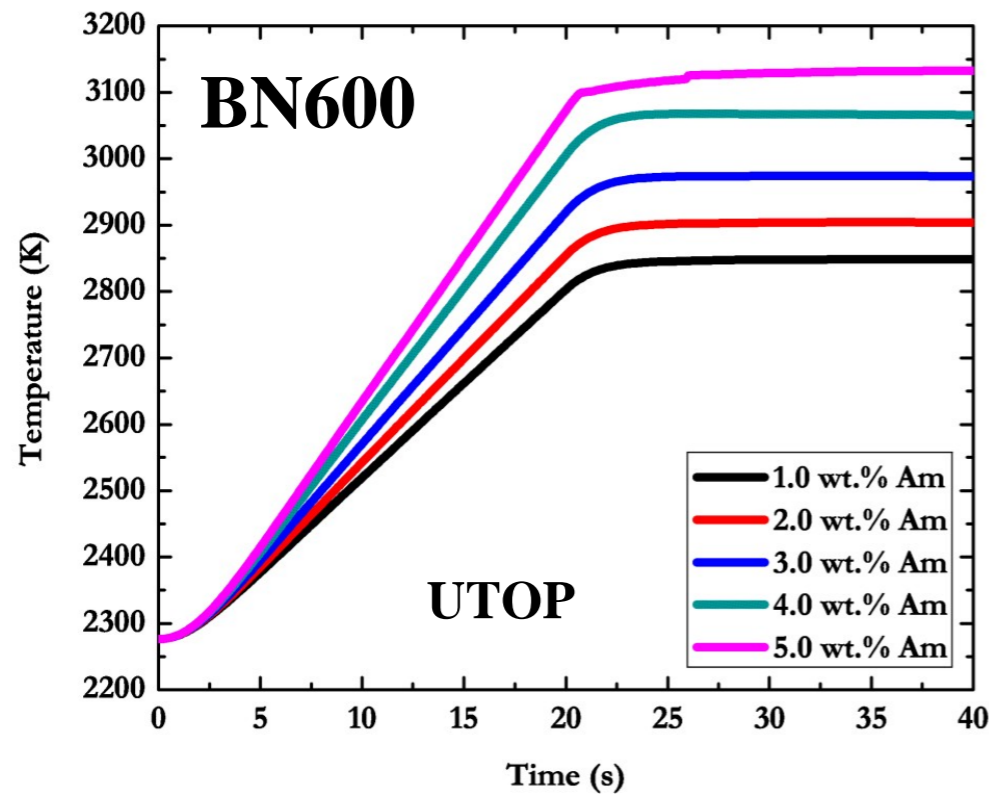




# Transient simulation results



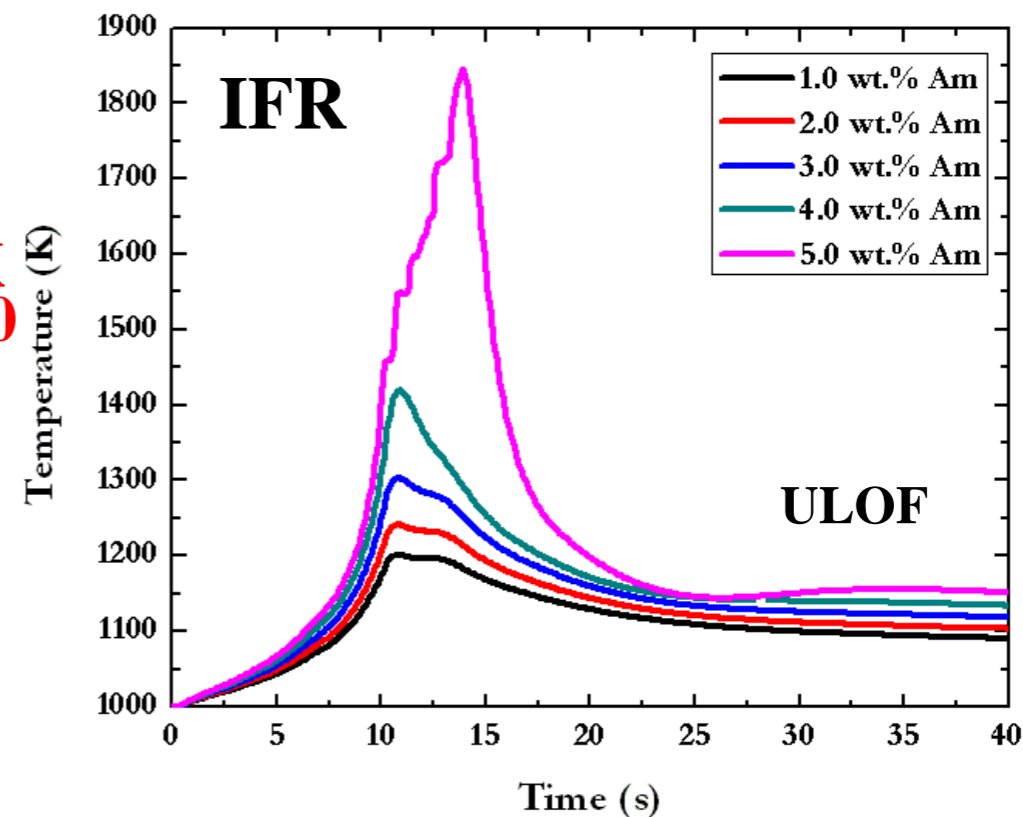
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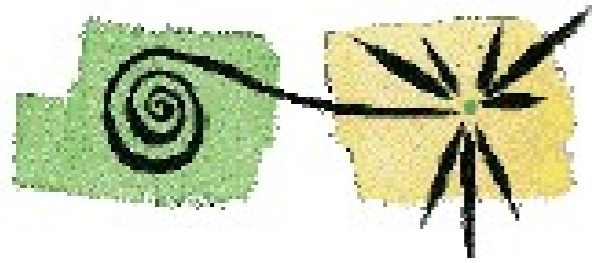


- Fuel temperature at hot spot increases by 300 K when Am content increases from 1.0 wt.% to 5.0 wt.% during UTOP.
- 1.0 wt.% additional Am leads to 6 % power penalty, preserving 100 K margin to fuel melting.

➤ Fuel temperature at hot spot increases by 700 K when Am content increases from 1.0 wt.% to 5.0 wt.% during ULOF.

➤ 1.0 wt.% additional Am leads to 10 % power penalty, preserving 100 K margin to fuel melting.





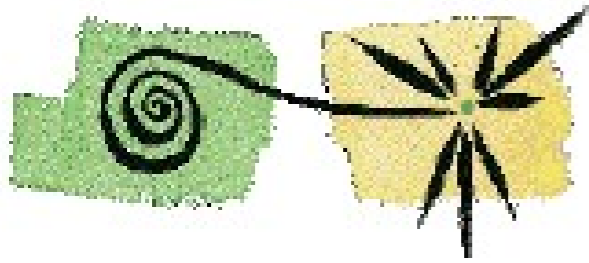
# Conclusions



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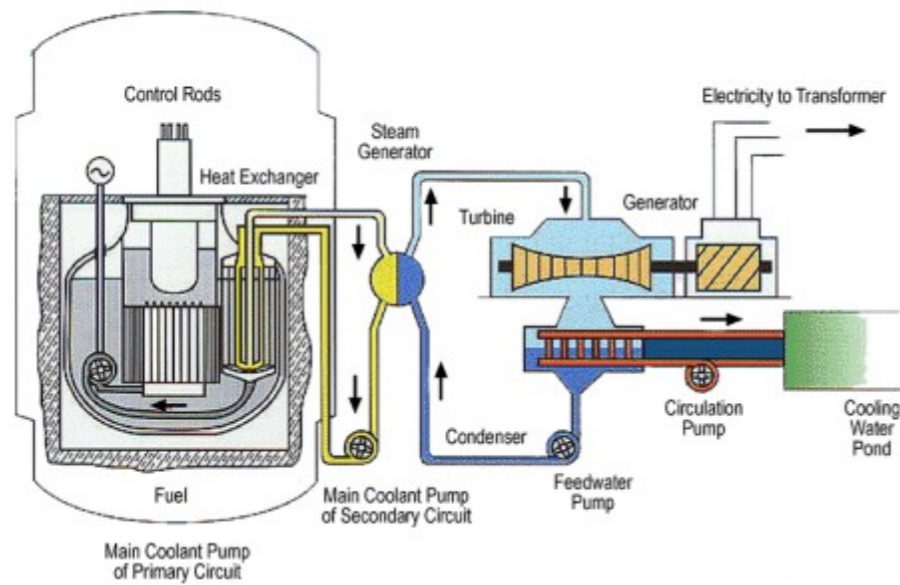
- ❖ **Transmutation of Am in SFRs loaded with oxide and metallic fuel may be possible but leads to significant power penalty.**
- ❖ **Fuels with high thermal conductivity (e.g., nitride fuel) and coolant with high boiling point (e.g., liquid Pb) could be considered.**
- ❖ **ADS is more efficient and suitable for Am transmutation.**





*Thanks for listening !  
Questions are welcomed!*

**BN-600 Reactor Design**



95: Americium      2,8,18,  
                                 32,25,  
                                 8,2

